

Test reactor irradiations in support of power reactor fuel R&D

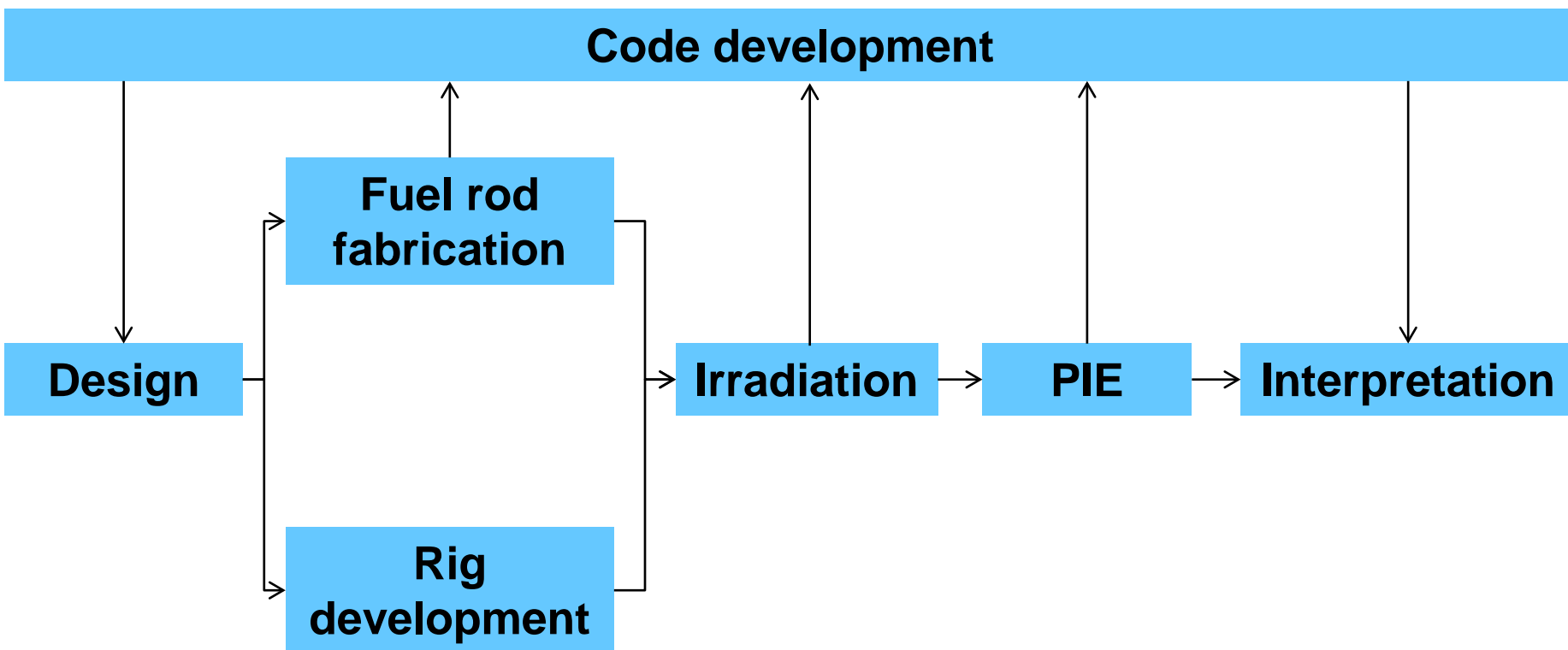
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STUDIECENTRUM VOOR KERNENERGIE
CENTRE D'ETUDE DE L'ENERGIE NUCLEAIRE

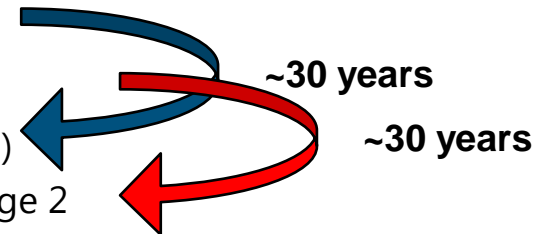
Nuclear fuel research: an integrated approach



- “Licensing support” R&D
 - Context
 - Example: CHIPS program on (U,Pu)O₂ fuel (2006-today)
- “Exploratory” R&D
 - Context
 - Example: (Th,Pu)O₂ research (2001-today)

Context of “licensing support R&D”

- Some dates:
 - 1958: first MOX production (SCK•CEN)
 - 1963: first LWR MOX irradiation (BR3)
 - 1986: first industrial LWR MOX production (BELGONUCLEAIRE)
 - 1994: first MOX loading in the Belgian NPPs Doel 3 and Tihange 2
- Between 1986-2005, clad development and MOX licensing programs were conducted in BR-2 (clients: fuel vendors, utilities, licensing authorities):
 - Cladding: ZIRLO, Low-Sn Zr-alloys, duplex Zr-alloys
 - MOX: Typical Pu enrichment: 4-8 wt% Pu/(U+Pu)
 - BU range: 20-80 GWd/tHM
 - Base irradiation in PW loop & transient testing in PW capsule
- In 2006, Japan's Nuclear Energy Safety organization (JNES) contracted two licensing support R&D programs (extend the enrichment range - broaden the high BU exp. basis) :
 - In Halden (HBWR Norway) and CEA (France), experimental MOX of ultra-high BU was studied
 - In SCK•CEN, commercial MOX with a bounding* enrichment of 14% was studied (“CHIPS”)

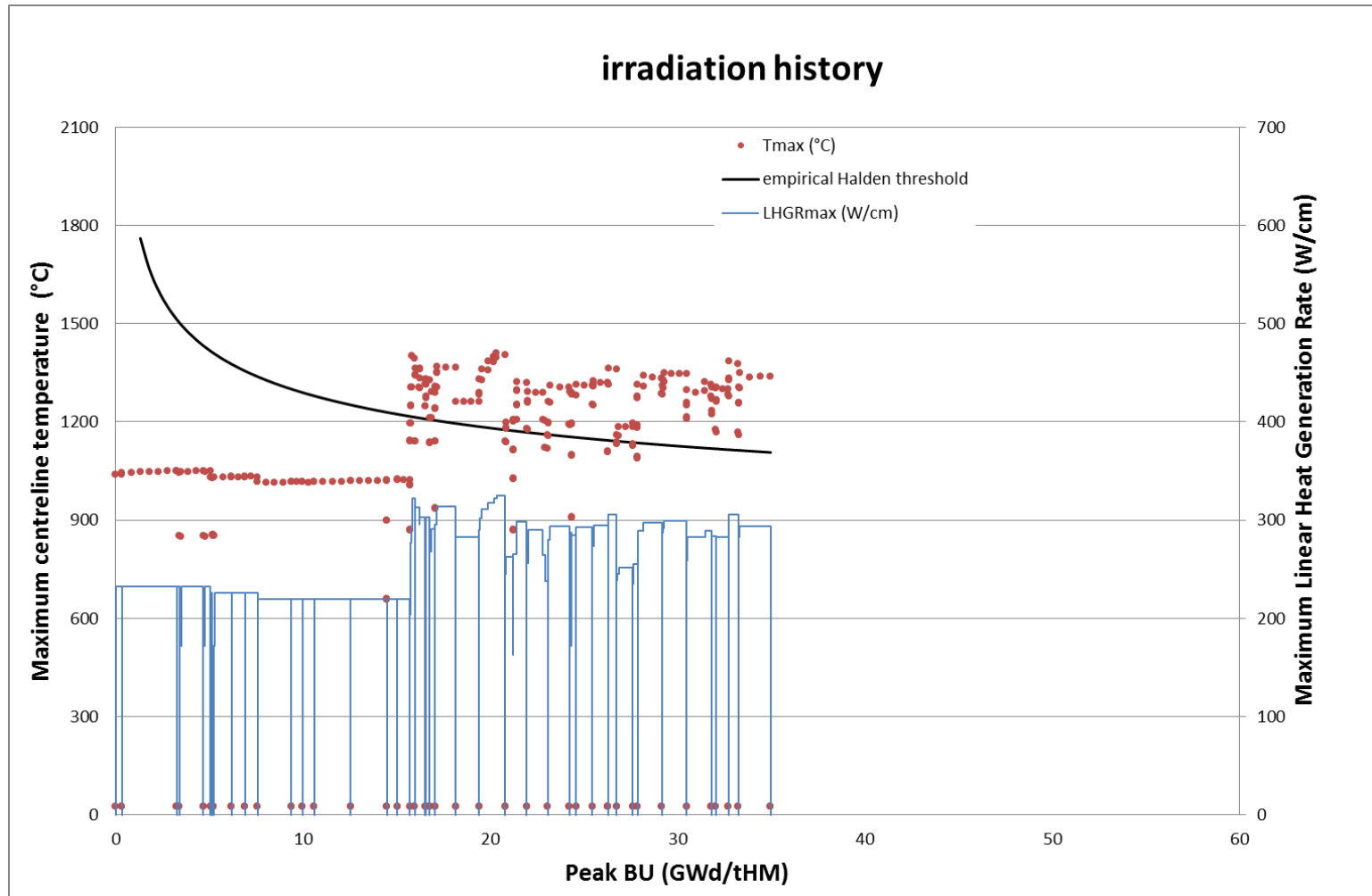


(*) bounding for the licensing applications considered in the Pluthermal plan

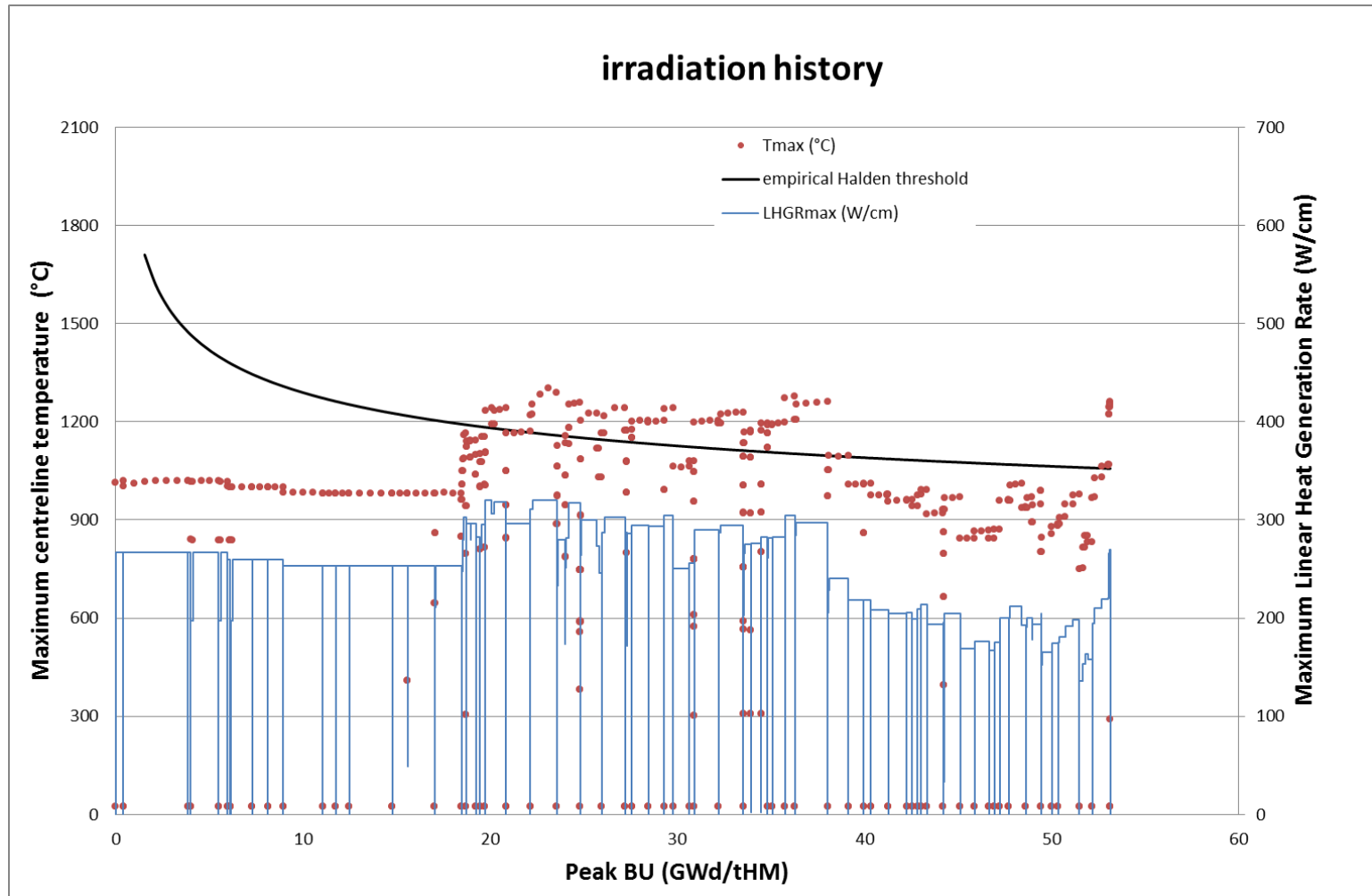
Objectives of the CHIPS program

- In 2006, in view of the delay of their FBR program and to reduce the growing Pu stockpiles, Japan decided to utilize Pu in 1/3rd of their LWR reactor fleet
- Japan's licensing support organization decided to develop experimental datasets (independent of vendor or utility-owned datasets)
- Test irradiations and detailed post-irradiation investigations:
 - 35 GWd/tHM, high rating, no transient
 - 53 GWd/tHM, high rating, transient at EOI
 - 55 GWd/tHM, low rating, transient at MOI and EOI

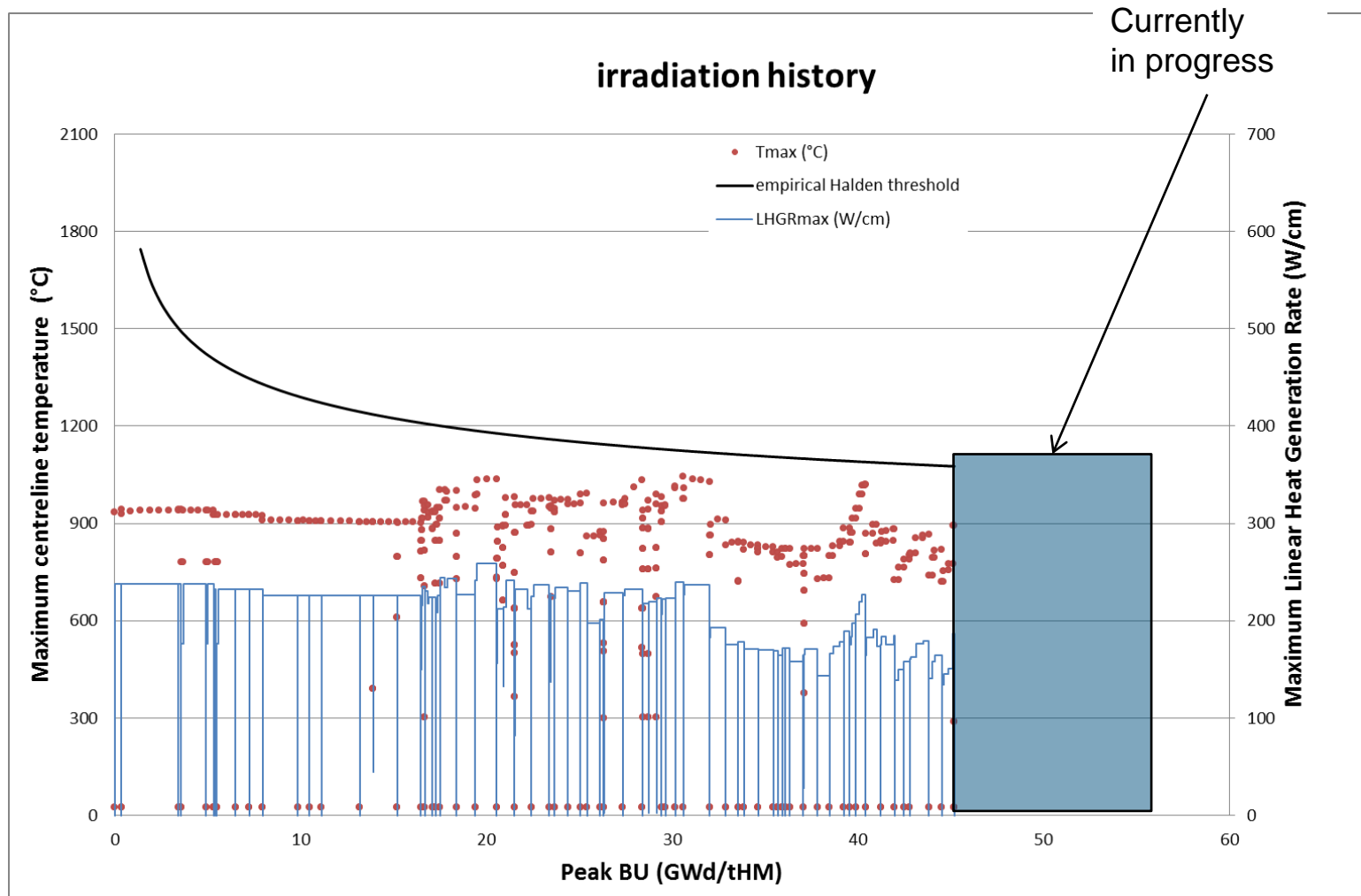
High power, medium burnup



High power, high burnup



Low power, high burnup

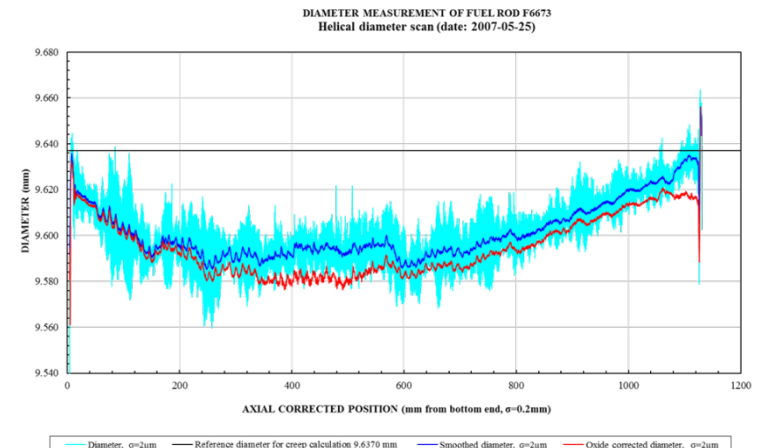
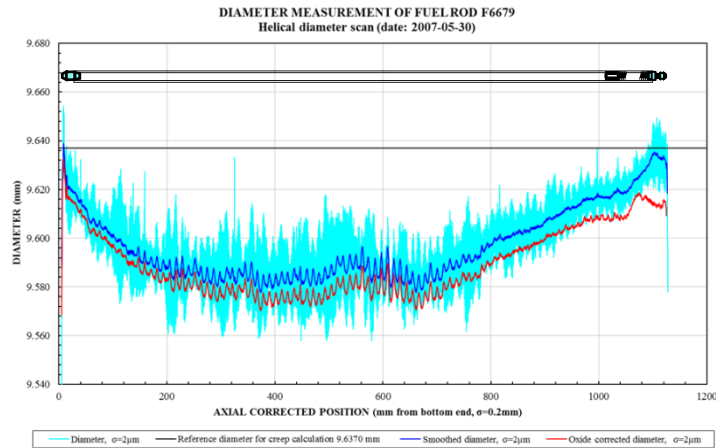


Comparison of PCMI

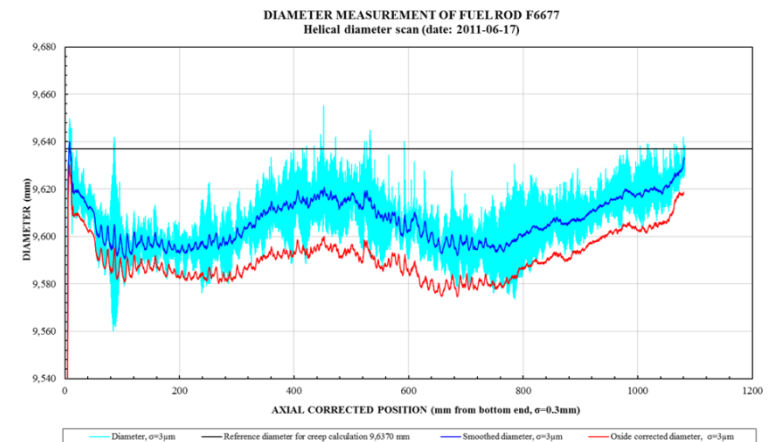
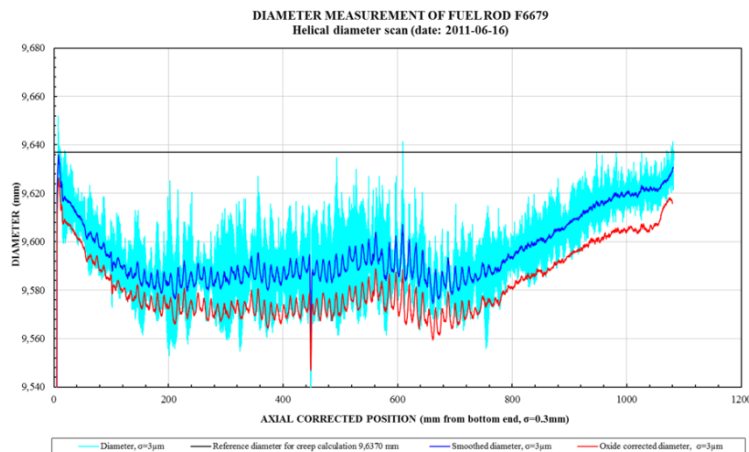
Low rating

High rating

Medium BU



High BU



Comparison of fuel behaviour

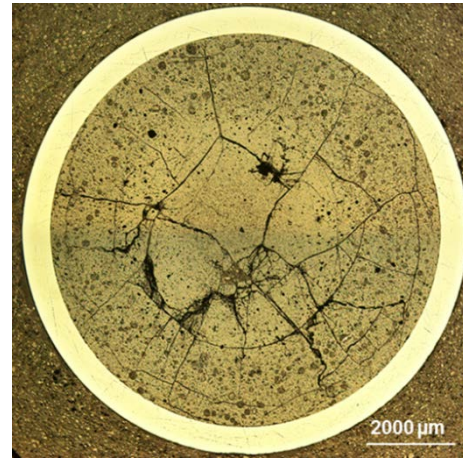
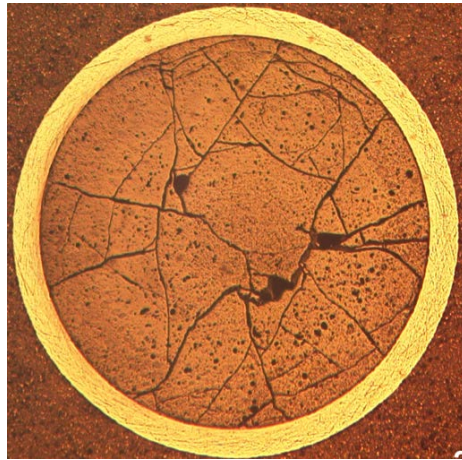
CT1, 35

Rod 1

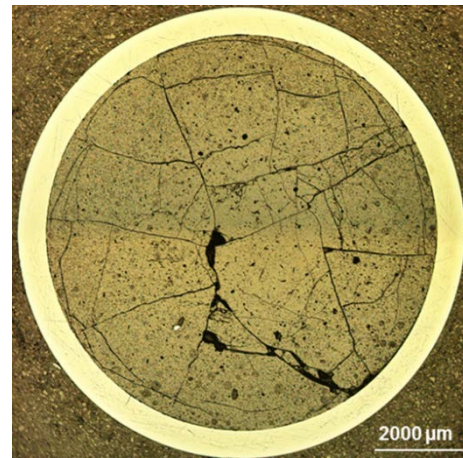
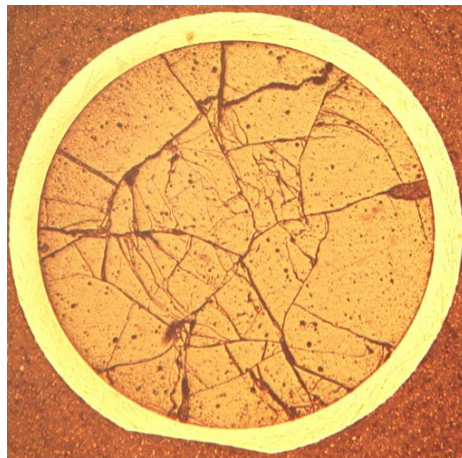
Rod 2

CT3, 52

High rating



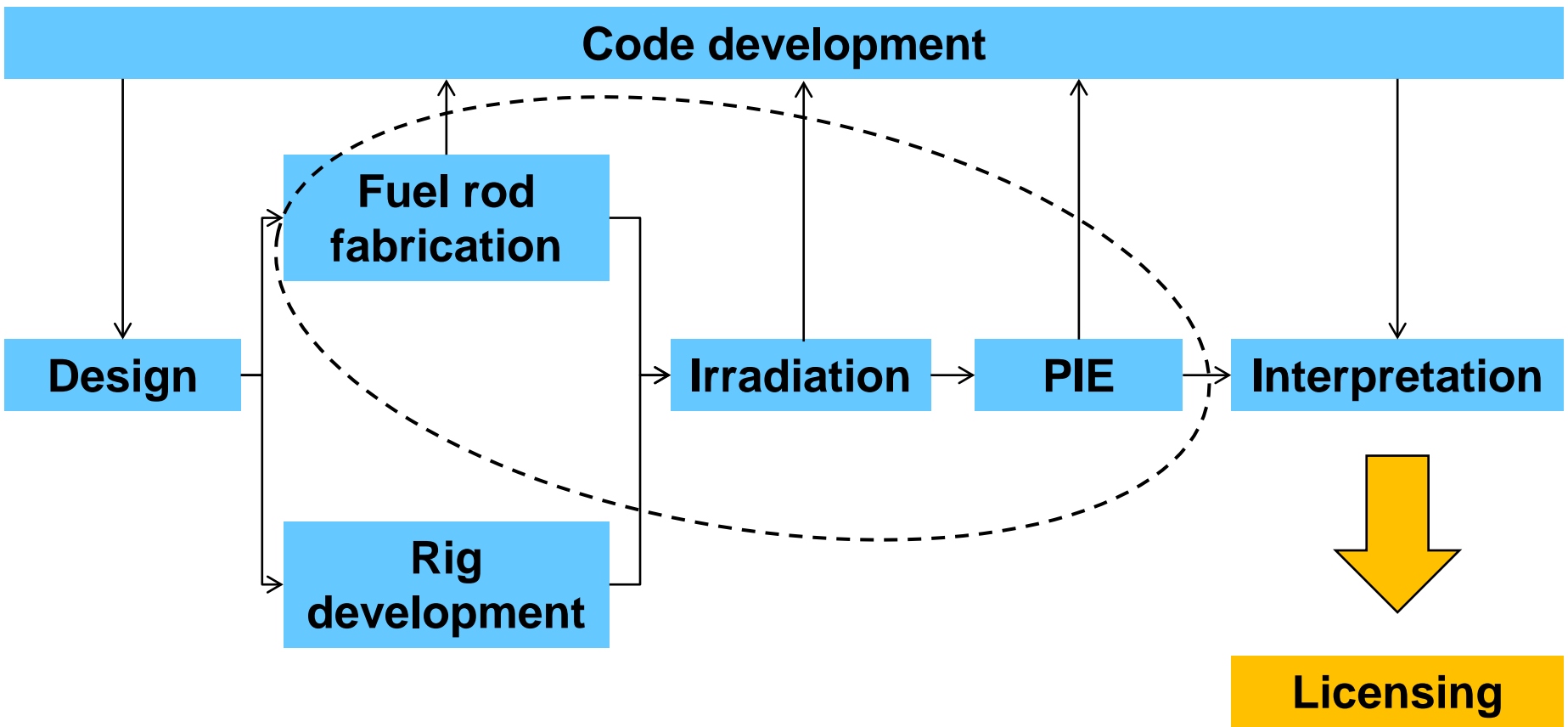
Low rating



CT2b, 25

CT5, 41

Outcome of licensing support program



- “Licensing support” R&D
 - Context
 - Example: CHIPS program on (U,Pu)O₂ fuel (2006-today)
- “Exploratory” R&D
 - Context
 - Example: (Th,Pu)O₂ research (2001-today)

Alternatie
vuur"

Jonathan Witterman - 08/01/13

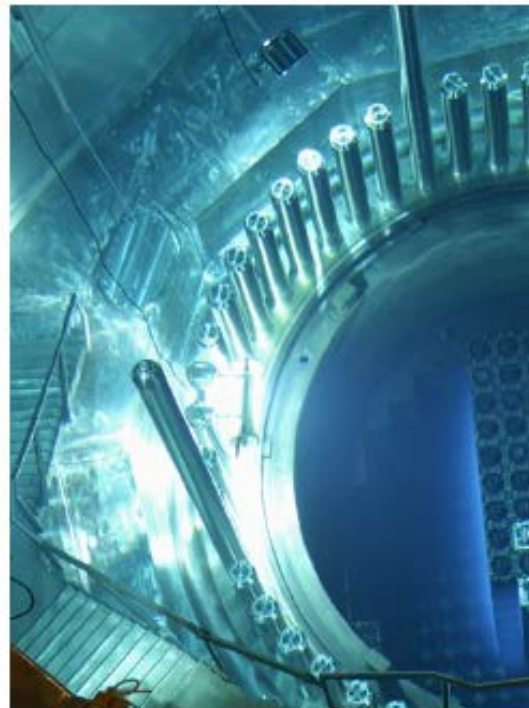


© belga. De kerncentrale van D

Terwijl veel Europese
tijdperk vaarwel zegg
ontwikkeling van een
thorium, door voorst
energiedoorbraak sin

Dat is misschien wat
overdreven, zegt Geoff
kernenergie-expert van
universiteit van Cambr

Thorium: alternatief v



Door Hans Labohm 22 oktober 2014

Volkskrant.nl

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Azië werkt aan energiewonder van thorium

Door: Jonathan Witterman - 08/01/13, 06:27



© AFP. De Verenigde Staten bij nacht, op een foto van Nasa. Thorium zou in potentie de hele wereld tienduizenden jaren van elektriciteit kunnen voorzien.

China, Japan en India hebben zich gestort op kernenergie uit het metaal thorium. De gevolgen kunnen revolutionair zijn.

Terwijl veel Europese landen na de Fukushima-ramp het nucleaire tijdperk vaarwel zeggen, storten China, Japan en India zich op de ontwikkeling van een alternatief: kernenergie uit het metaal thorium, door voorstanders al aangekondigd als 'grootste energiedoorbraak sinds het vuur'.

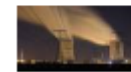
“ In potentie is er voldoende thorium om de hele wereld tienduizenden jaren van elektriciteit te voorzien.

Jan Leen Kloosterman, reactorfysicus van de Technische Universiteit Delft

Foto: Thorium reactor

MEER OVER

Kernenergie | Energie | Japan | Wetenschap



Reactorblok kerncentrale afgesloten na brand



Kerncentrale Californië gesloten



Belgische kernreactoren

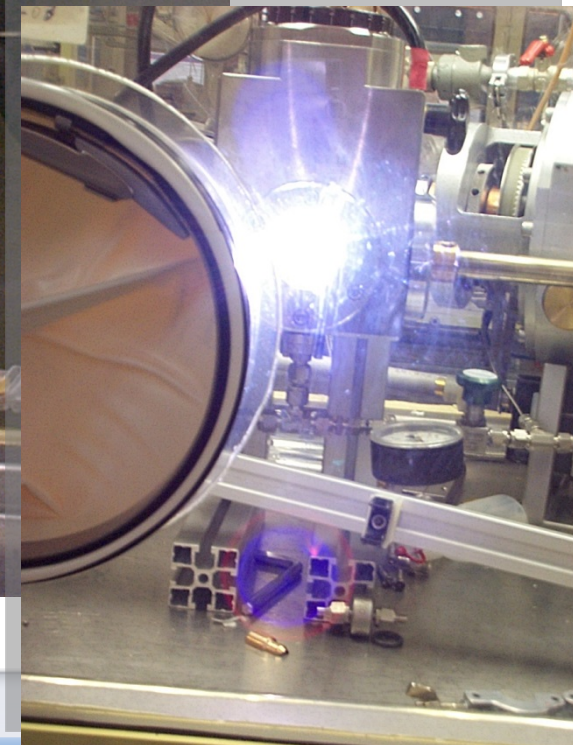
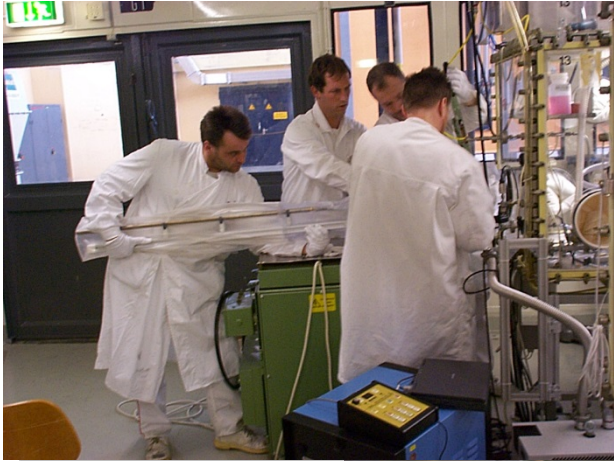


Geen haarscheurtjes in Borssele

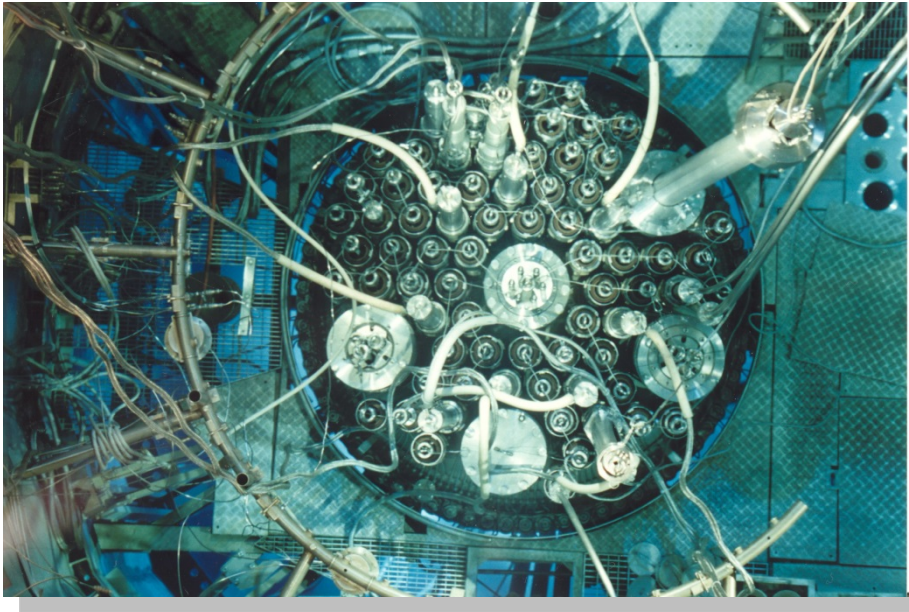
- Experimental Thorium program (2001-today)
 - Part I - Integral (Th,Pu)O₂ programs
 - Part II - Fabrication and solid state research
 - Outlook: where do we stand and what's next?

OMICO (2001-2007)

Instrumented pin assembly at ITU



Assembly & irradiation: TCH & BR-2

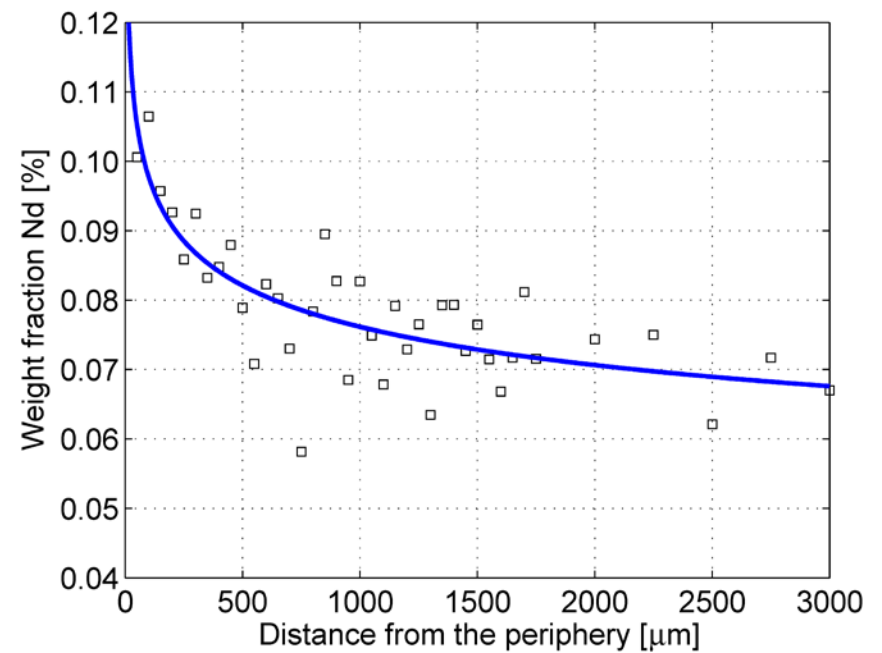
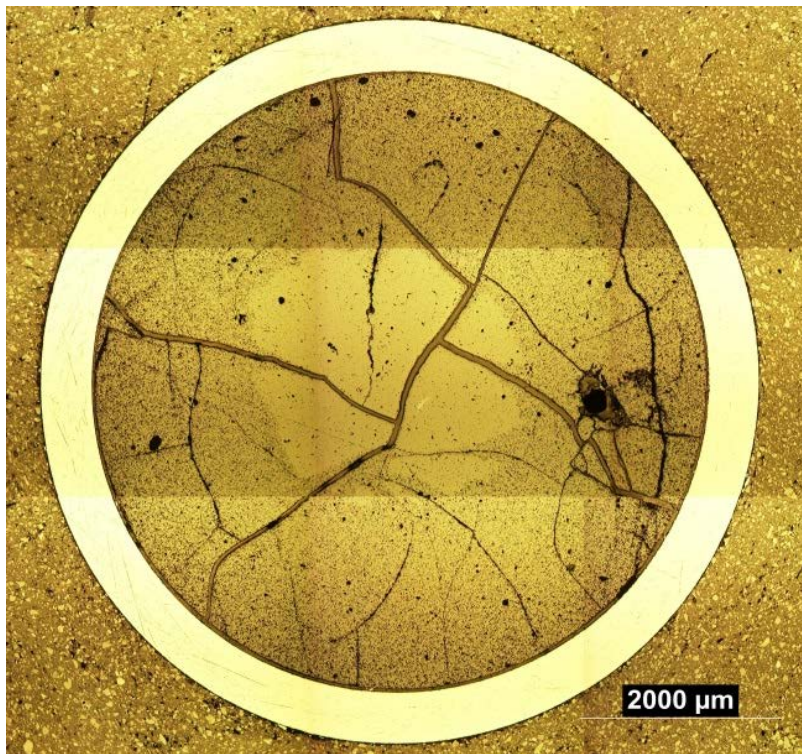


OMICO ready to be transferred to BR-2
(29-10-2004)

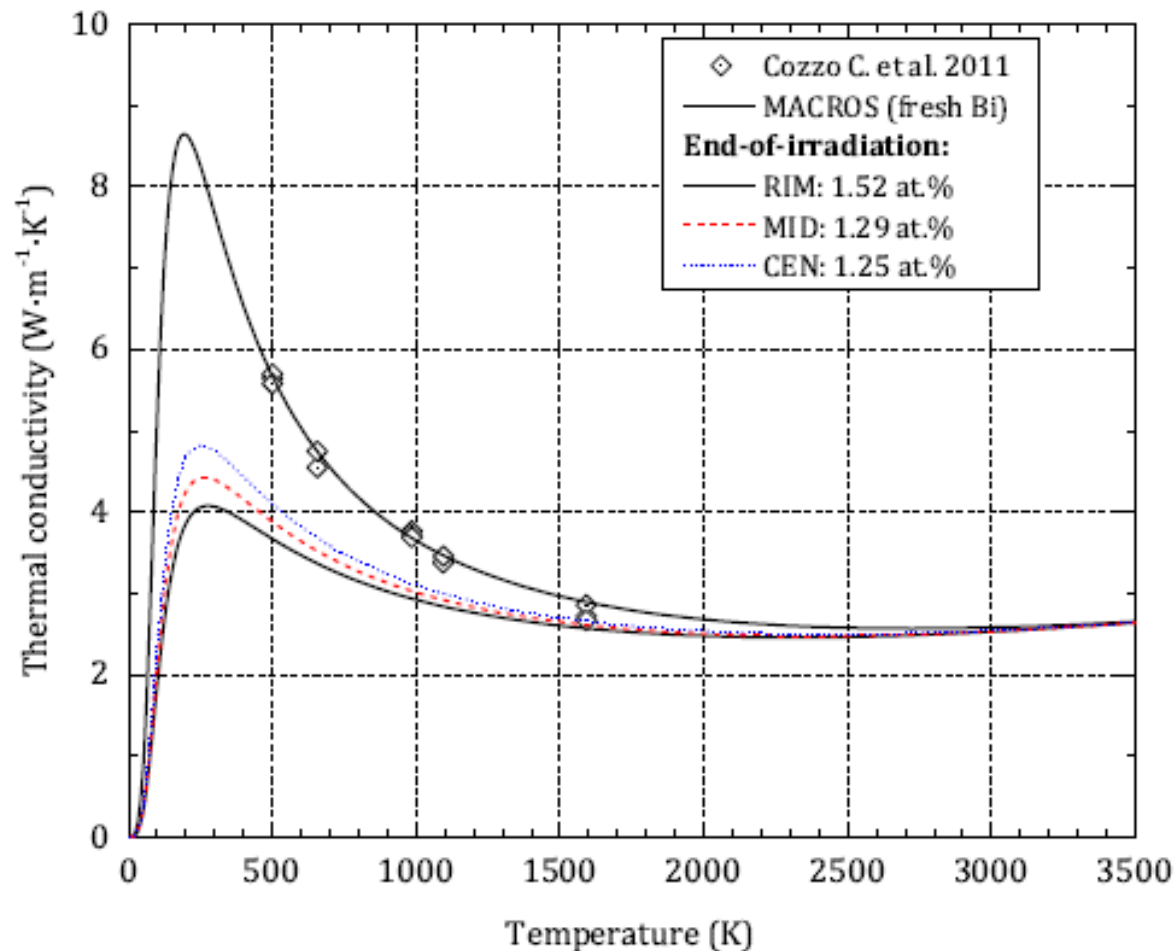


LWR-DEPUTY (2006-2011)

Post-irradiation examinations of (Th,Pu)O₂



(Th,Pu)O₂: Thermal conductivity measurements & model



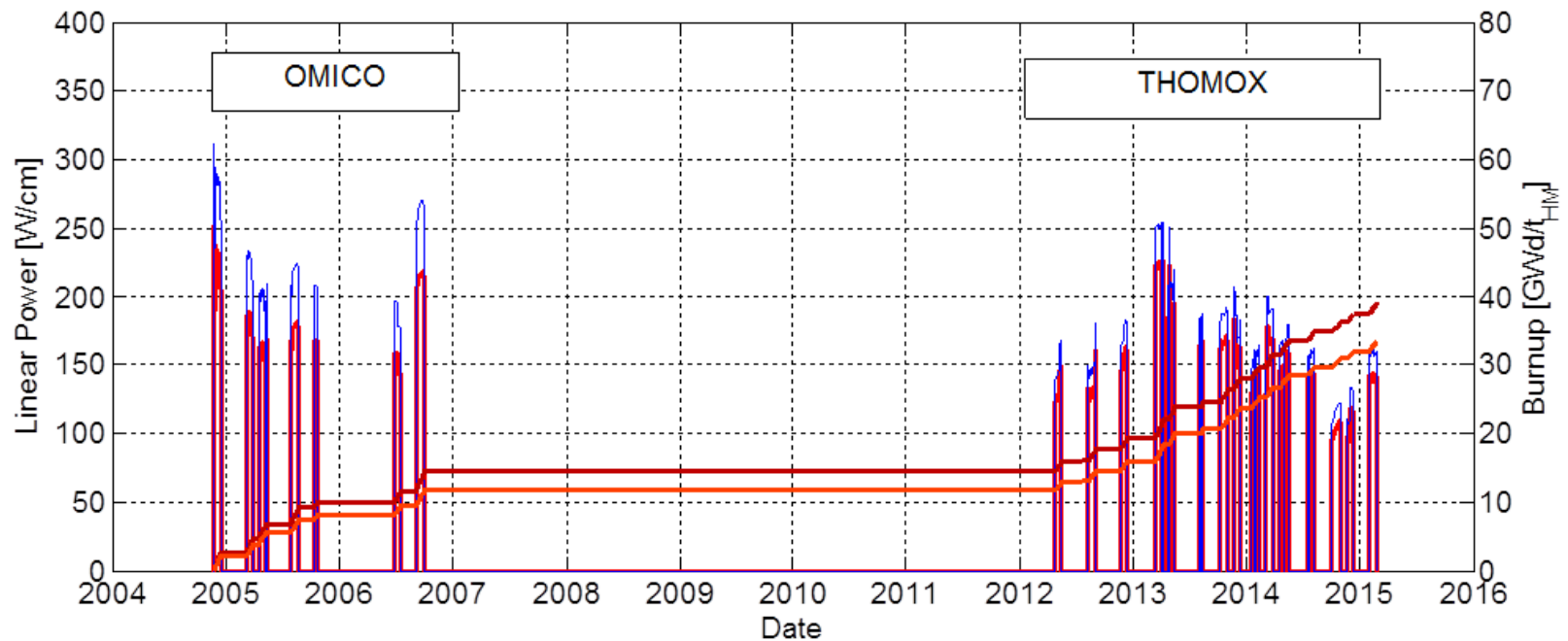
THOMOX (2012-2015)

In-reactor performance of (Th,Pu)O₂ in BR2 Irradiations 2004-2015

Highest linear power (W/cm): 311.4

Final burnup <BU> (GWd/t_{HM}): 33.6

Final burnup BU_{peak} (GWd/t_{HM}): 39.2



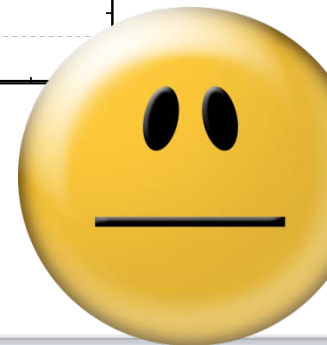
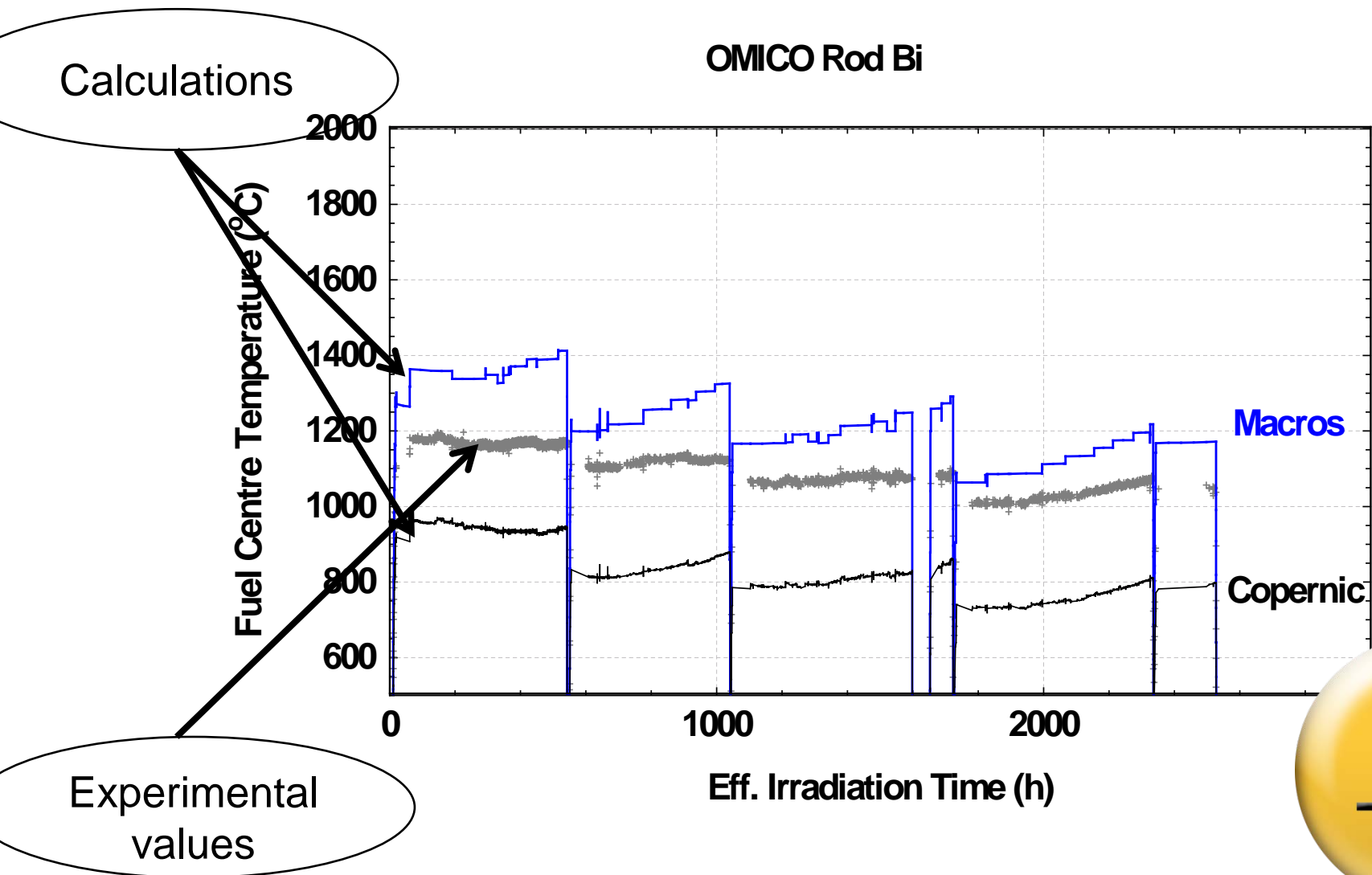
Non-instrumented (Th,Pu)O₂ irradiations

Objective: reach ca. 40 GWd/t_{HM} (industrially-relevant burnup)

What did we learn?

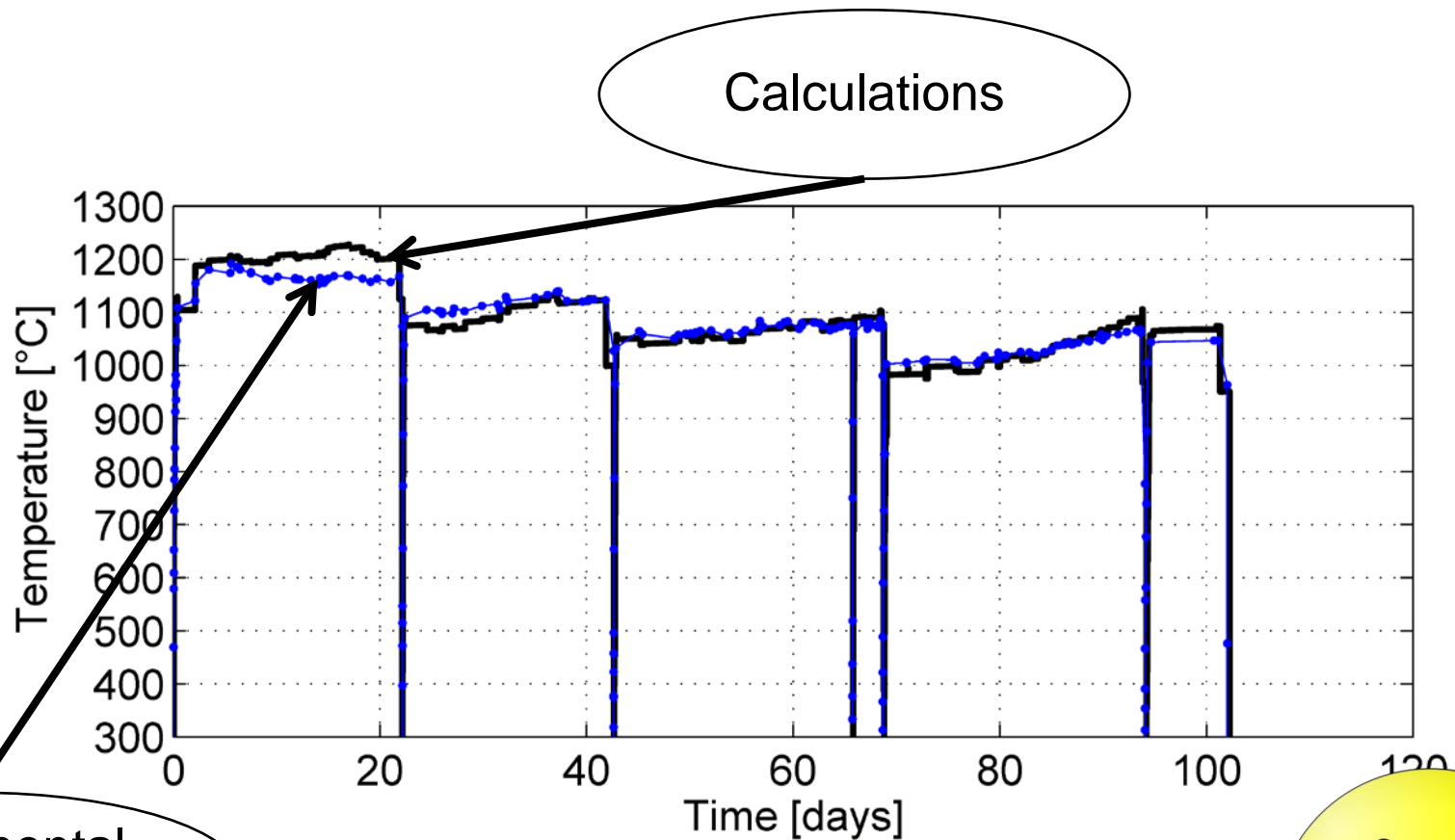
In-reactor performance of (Th,Pu)O₂ in BR2

Model predictions (2007) versus experiment: SCK•CEN & AREVA



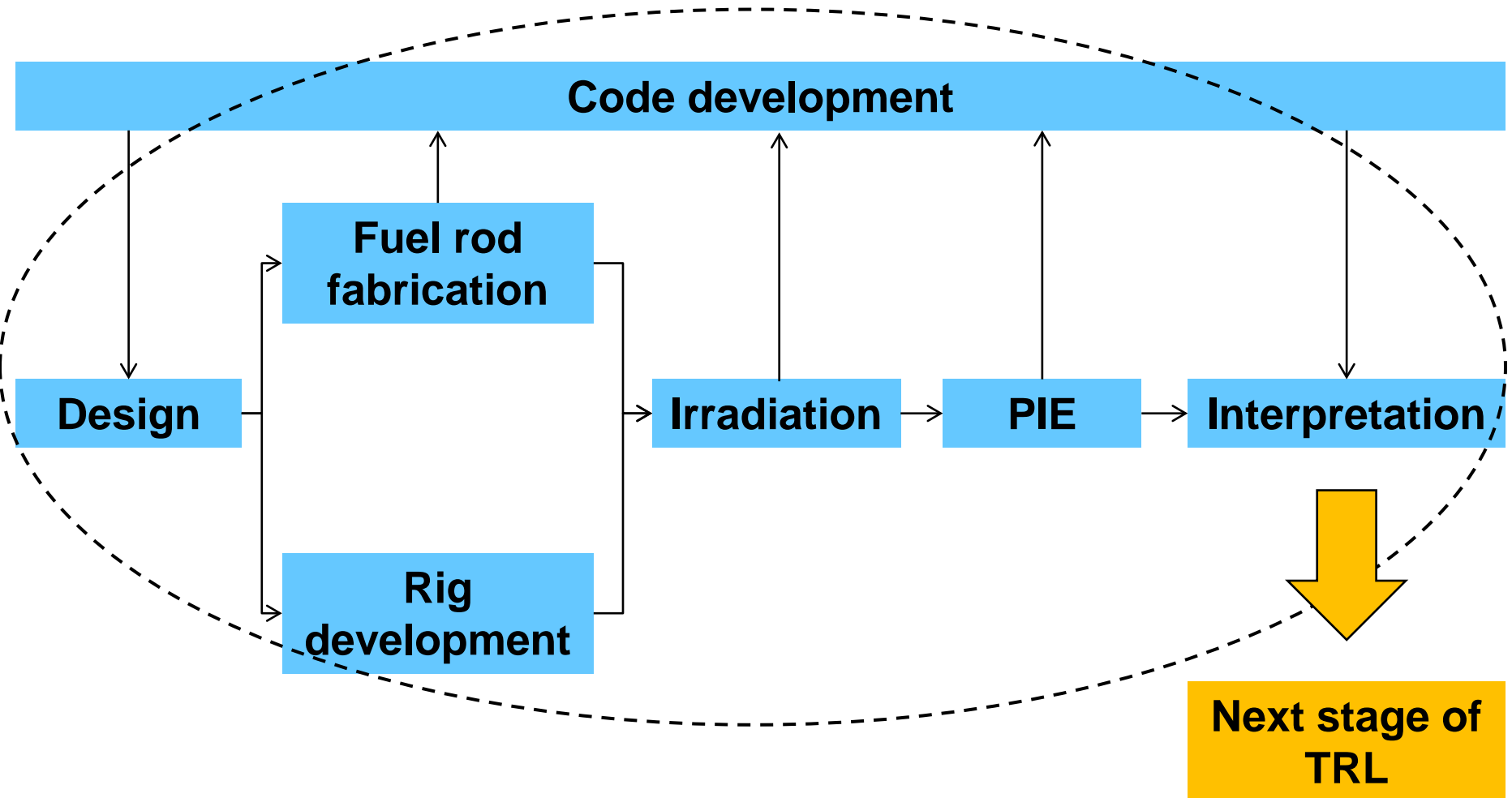
In-reactor performance of (Th,Pu)O₂ in BR2

Model predictions (2015) versus experiment



B. Boer, S. Lemehov, M. Wéber, Y. Parthoens, M. Gysemans, J. McGinley, J. Somers, M. Verwerft, *Irradiation performance of (Th,Pu)O₂ fuel under Pressurized Water Reactor conditions*, J. Nucl. Mater., 471 (2016) 97-109.

Outcome of exploratory R&D



Thorium and thorium-plutonium fuel R&D

Where do we stand today?

- **Performance qualification of (Th,Pu)O₂ has been achieved**

- **Open issues:**

- Elaborate production routes that *can be implemented industrially*
- *Demonstrate the in-reactor performance in a power plant*
- *Quantify the fuel back-end options*

Table 1
Proposed application of technical readiness levels to reactor fuel development and qualification

TRL	TRL Function	Generic Definition	Fuel Development-Specific Definition	Fuel Dev. Phase
1			Technical review leading to a preliminary selection of options for further development	1
2	Technology Down-Selection	Technology concepts and/or applications formulated	Fuel candidates selected from options, based on selection criteria	1
3		Analytical and experimental demonstration of critical function and/or proof of concept	Calculational analysis and lab-scale experimentation and characterization addressing feasibility, including: fabrication process development, property measurement, and ex-pile tests	1
4	Final Process Design and Integration	Component and/or bench-scale validation in a laboratory environment	Establish proof of concept. Fabrication of representative components (e.g., fuel pins) and irradiation in a laboratory environment. Performance phenomena identified with irradiation tests	2
5		Component and/or bench-scale validation in a relevant environment	Irradiation testing of prototypes in a relevant environment (e.g., neutron densities, fuel and cladding temperatures, cladding damage rates) is performed and assessed	2
6	Full-scale integrated testing	System/subsystem model or prototype demonstration in relevant environment	Prototypic rod/compact and assembly/element irradiation in representative environment, under full range of relevant normal and off-normal conditions. Representative compositions Design parameters investigated Information is sufficient to support a Fuel Specification and a Fuel Safety Case (which, in turn, support larger System Demonstration to achieve TRL7)	3
7		System prototype demonstration in prototypic environment	Fabrication of reference fuel derived from production supply sources irradiated to design conditions and utilization Irradiation in representative environment Prototypic design, Prototypic fabrication processes, Representative compositions	4
8	Full-scale demonstration	Actual system completed and qualified through test and demonstration		
9		Actual system proven through successful mission operations		

Screening

Pre-qualification

Qualification

Demonstration

- Experimental Thorium program (2001-today)
 - Part I - Integral (Th,Pu)O₂ programs
 - Part II - Fabrication and solid state research
 - Outlook: where do we stand and what's next?

Solid state research and fabrication aspects of ThO₂

- *"UO₂ and ThO₂ doped nuclear fuel"* (A. Baena, 2011-2016 PhD thesis)
 - Solid state study on the fluorite structure effects upon doping with different lanthanides
 - Associated master thesis / internships:
 - *"Advanced manufacturing routes for ThO₂ doped with lanthanides"* (M. Pasniewsky)
 - *"Synthesis of ThO₂ doped with Gd via wet chemical routes"* (P. Zsabka)
 - *"Recycling of ThO₂ scrap from nuclear fuel pellets fabrication"* (A. Schneider, L. Gijsemans)
- *"Improving the sintering behaviour of ThO₂"* (T. Wangle, 2015-2019 PhD thesis, industrial sponsor: Solvay)
 - Converting thorium nitrate into sinterable oxide powders
 - Associated master thesis / internships:
 - *"Effect of pH, T and concentration on the oxalic precipitation of Th⁴⁺ from nitric acid media"* (M. Beliš)

- Experimental Thorium program (2001-today)
 - Part I - Integral (Th,Pu)O₂ programs
 - Part II - Fabrication and solid state research
 - Outlook: where do we stand and what's next?

- The next steps:
 - **Elaborating production routes that can be implemented industrially**
 - We're working on it ...
 - **Demonstrate in-reactor performance**
 - Qualification of neutronics codes: benchmarking and uncertainty analysis
 - Assembly design (neutronic) for given core management
 - AOO, DBA analysis
 - Perform lead irradiation tests (power reactor: normal operation, test reactor: off-normal conditions)
 - ... still a lot to do ...

Thorium and thorium-plutonium fuel R&D

Where do we stand today and where do we go?

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Proposed application of technical readiness levels to reactor fuel development and qualification

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4	Final Process Selections and integration	Component and/or bench-scale validation in a laboratory environment	Establish proof of concept. Fabrication of irradiation testing samples in accordance with QA requirements. Design parameters and features established. Performance phenomena identified with proof-of-concept irradiation testing	2
5		Component and/or breadboard validation in a relevant environment	Irradiation testing of prototypic rods/compacts under nominal representative conditions (e.g., fission densities, fuel and cladding temperatures, cladding damage rates) is performed and assessed	
6	Full-scale integrated testing	System/subsystem model or prototype demonstration in relevant environment	Prototypic rod/compact and assembly/element irradiation in representative environment, under full range of relevant normal and off-normal conditions. Representative compositions. Design parameters investigated. Information is sufficient to support a Fuel Specification and a Fuel Safety Case (which, in turn, support larger System Demonstration to achieve TRL7)	3
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9		Actual system proven through successful mission operations		

Screening

2001

Pre-qualification

2016

Qualification

Demonstration



2031